



Neutron & Gamma Correlations using CGM in MCNP 6.2

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Outline

- Reaction Sampling in MCNP
- Cascading Gamma-ray and Multiplicity (CGM)
- CGM Modifications in MCNP 6.2
- Examples
- Results
 - Secondary spectra comparisons
 - Correlated neutrons & gammas
- Conclusion

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Reaction sampling in MCNP

- Accomplished using ACE (A Compact ENDF) data libraries

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Reaction sampling in MCNP

- Accomplished using ACE (A Compact ENDF) data libraries
 - Reaction information (Cross-section data)

$$\sigma_i$$

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Reaction sampling in MCNP

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 - Reaction information (Cross-section data)
 - Secondary particle information

$$\sigma_i \quad \overline{\nu}_f \quad \overline{M}_n \quad \overline{M}_\gamma$$

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average

$$\overline{M}_{\gamma} = 1.4$$
$$M_{\gamma} = 1 \quad (60\%)$$
$$M_{\gamma} = 2 \quad (40\%)$$

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- MC sampling of secondary particles can be independent of incident reaction

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- MC sampling of secondary particles can be independent of incident reaction
 - Limits correlation of secondary particles

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Cascading Gamma-ray and Multiplicity (CGM)

- Originated from CoH code¹⁰

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Cascading Gamma-ray and Multiplicity (CGM)

- Originated from CoH code¹⁰
- Samples nuclei decay based on Hauser-Feshbach (HF) model⁹

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Cascading Gamma-ray and Multiplicity (CGM)

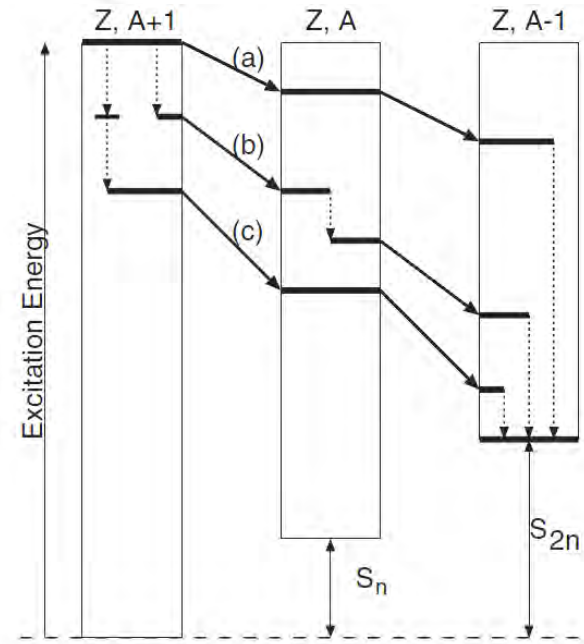
- Originated from CoH code¹⁰
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Cascading Gamma-ray and Multiplicity (CGM)

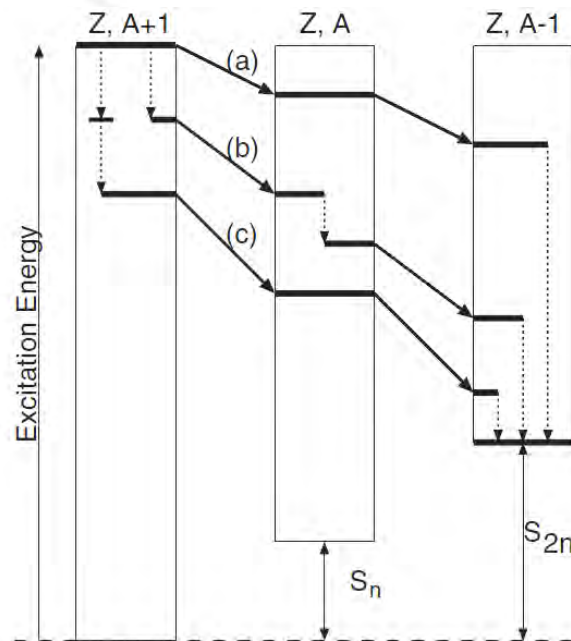
- Originated from CoH code¹⁰
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 - Passed Excitation energy (E_x) value



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Cascading Gamma-ray and Multiplicity (CGM)

- Originated from CoH code¹⁰
- Samples nuclei decay based on Hauser-Feshbach (HF) model⁹
- Implemented in MCNPX V2.7.0⁸
 - Passed Excitation energy (E_x) value
 - Calculate decay probabilities for neutrons & photons using Monte Carlo & deterministic methods



$$P(\varepsilon_n)dE_1 = \frac{T_n(E_x - S_n - E_1)\rho(Z, A - 1, E_1)}{N}dE_1$$

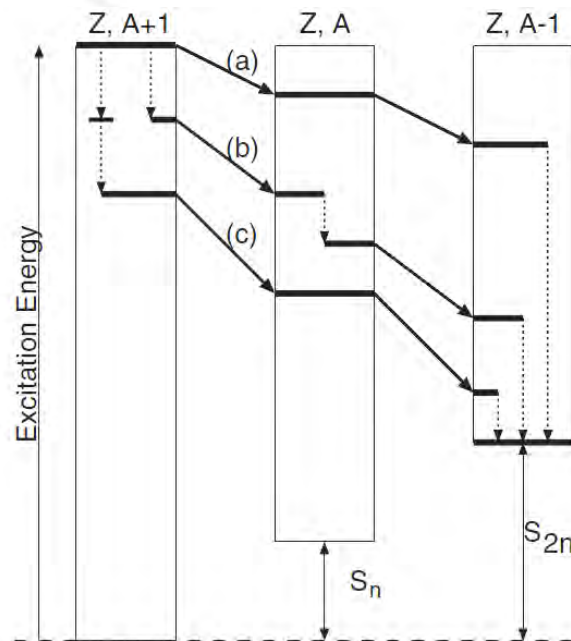
$$P(\varepsilon_\gamma)dE_0 = \frac{T_\gamma(E_x - E_0)\rho(Z, A, E_0)}{N}dE_0$$

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Cascading Gamma-ray and Multiplicity (CGM)

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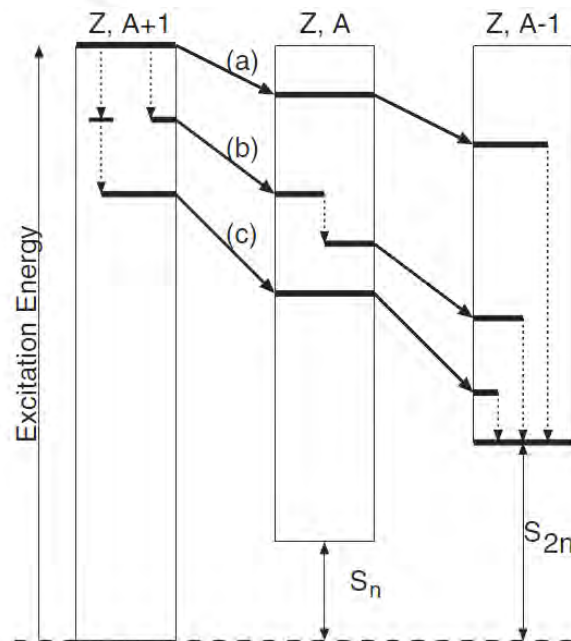
$$P(\varepsilon_\gamma)dE_0 = \frac{T_\gamma(E_x - E_0)\rho(Z, A, E_0)}{N}dE_0$$

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Cascading Gamma-ray and Multiplicity (CGM)

- Originated from CoH code¹⁰
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- Implemented in MCNPX V2.7.0⁸
 - Passed Excitation energy (E_x) value
 - Calculate decay probabilities for neutrons & photons using Monte Carlo & deterministic methods
 - Returned secondary gammas
 - Provided ability to correlate secondary gammas¹¹



$$P(\varepsilon_n)dE_1 = \frac{T_n(E_x - S_n - E_1)\rho(Z, A - 1, E_1)}{N}dE_1$$

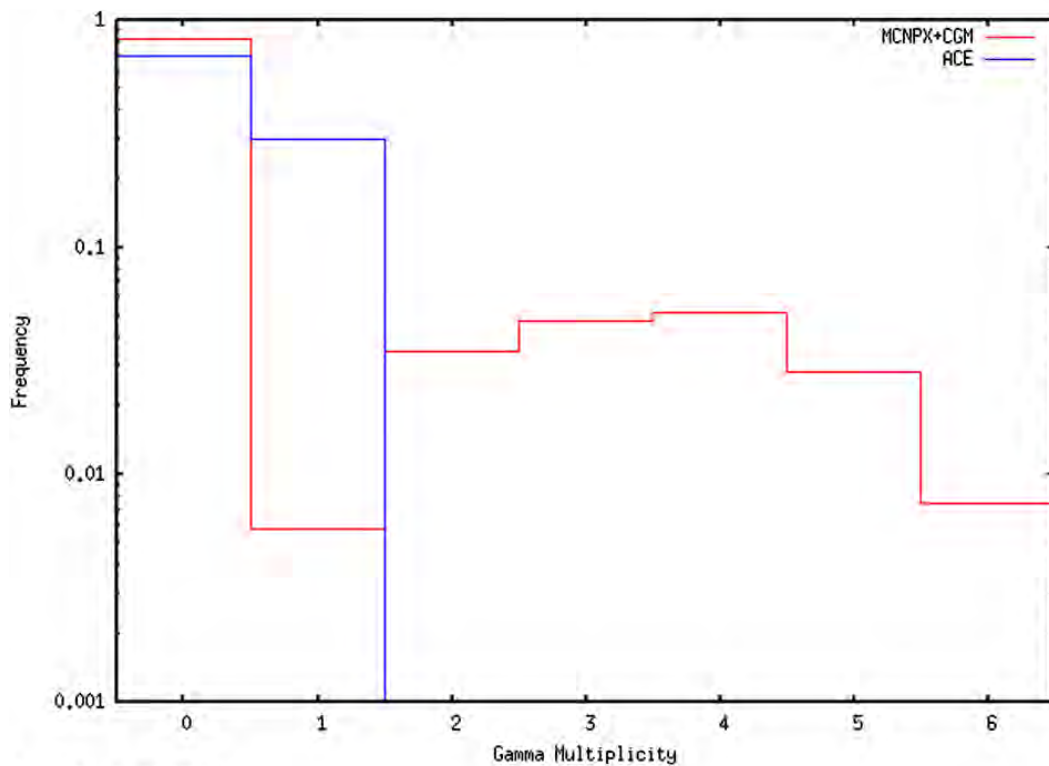
$$P(\varepsilon_\gamma)dE_0 = \frac{T_\gamma(E_x - E_0)\rho(Z, A, E_0)}{N}dE_0$$

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Cascading Gamma-ray and Multiplicity (CGM)

20 MeV, ^{56}Fe



6.49210, 0.000981258

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CGM Modifications in MCNP 6.2

- Modified to return neutrons

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CGM Modifications in MCNP 6.2

- Modified to return neutrons
- Excitation energy (E_x) passed to CGM has changed:

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CGM Modifications in MCNP 6.2

- Modified to return neutrons
- Excitation energy (E_x) passed to CGM has changed:

$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} - \sum_{i=1}^2 \frac{A_i}{A_i + m_i} Elab_i + Q$$

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CGM Modifications in MCNP 6.2

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Incident particle
energy

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Secondary neutron energies

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$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} - \sum_{i=1}^2 \frac{A_i}{A_i + m_i} Elab_i + \boxed{Q} \quad \text{Reaction Q-value}$$

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$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} + BE$$

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CGM Modifications in MCNP 6.2

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$$E_x = \boxed{\frac{A_{In}}{A_{In} + m_{In}} Elab_{In}} + BE$$

Incident particle energy

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CGM Modifications in MCNP 6.2

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$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} - \sum_{i=1}^2 \frac{A_i}{A_i + m_i} Elab_i + Q$$

$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} + \boxed{BE}$$

Neutron binding
energy

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$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} + BE$$

- Neutron captures forced to analog

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CGM Modifications in MCNP 6.2

- Modified to return neutrons
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$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} - \sum_{i=1}^2 \frac{A_i}{A_i + m_i} Elab_i + Q$$

$$E_x = \frac{A_{In}}{A_{In} + m_{In}} Elab_{In} + BE$$

- Neutron captures forced to analog
- MCNP6 – 9th entry on PHYS:N card
- MCNPX – 8th entry on PHYS:N card

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Examples

- Thermal neutron, high Z (1 eV, ^{207}Pb)
- Fast neutron, high Z (16 MeV, ^{207}Pb)

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Correlating secondary particles in MCNP

1. Activate ACE or CGM for neutron interactions

```
PHYS:n 8j 2 $ CGM  
c PHYS:n 8j 1 $ ACE
```

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Correlating secondary particles in MCNP

1. Activate ACE or CGM for neutron interactions
2. Create an F1 tally for neutrons & photons

```
PHYS:n 8j 2 $ CGM  
c PHYS:n 8j 1 $ ACE  
f11:n 1  
f21:p 1
```

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Correlating secondary particles in MCNP

1. Activate ACE or CGM for neutron interactions
2. Create an F1 tally for neutrons & photons
3. Use the pulse-height light (PHL) tally option with an F8 tally to pair the neutron & gamma F1 tally for coincidence counting

```
PHYS:n 8j 2 $ CGM  
c PHYS:n 8j 1 $ ACE  
f11:n 1  
f21:p 1  
f8:n,p 1  
ft8 PHL 1 11 1 1 21 1 0
```

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Correlating secondary particles in MCNP

1. Activate ACE or CGM for neutron interactions
2. Create an F1 tally for neutrons & photons
3. Use the pulse-height light (PHL) tally option with an F8 tally to pair the neutron & gamma F1 tally for coincidence counting
4. Bin the tally results by the # of neutrons and gammas produced

```
PHYS:n 8j 2  $ CGM
c PHYS:n 8j 1  $ ACE
f11:n 1
f21:p 1
f8:n,p 1
ft8 PHL 1 11 1 1 21 1 0
e8 0.5 1.5 2.5 3.5
fu8 0.5 1.5 2.5 3.5 4.5 5.5 6.5
      7.5 8.5 9.5 10.5 11.5 12.5
fq8 u e
```

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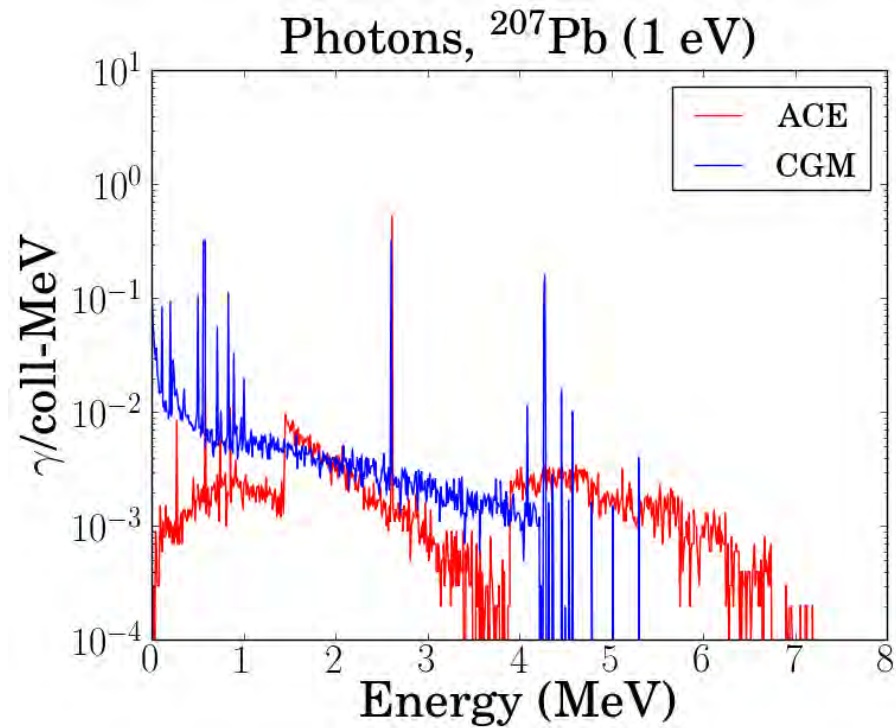
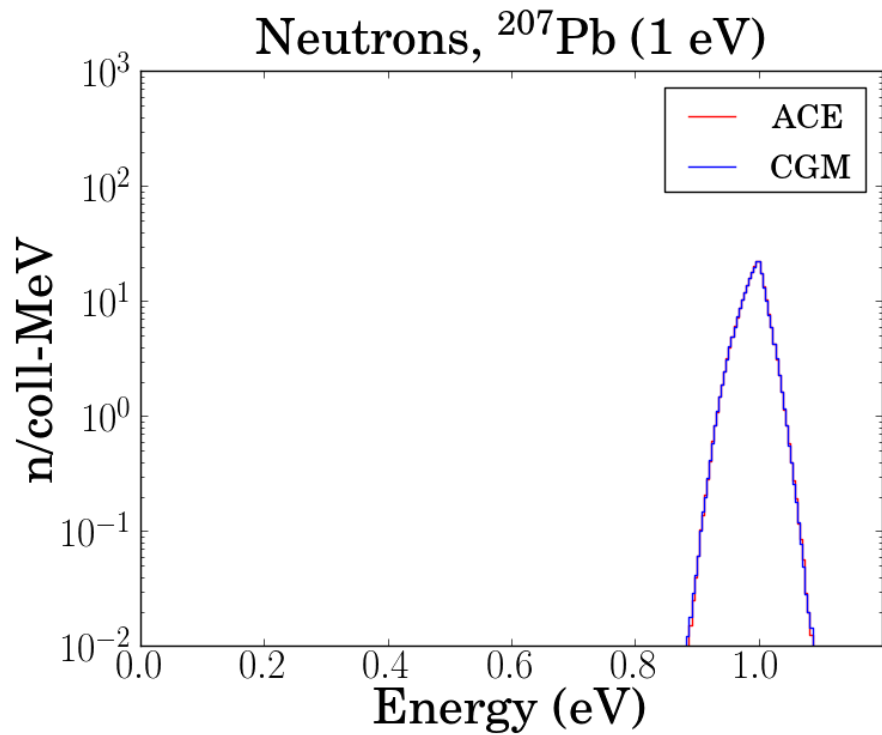
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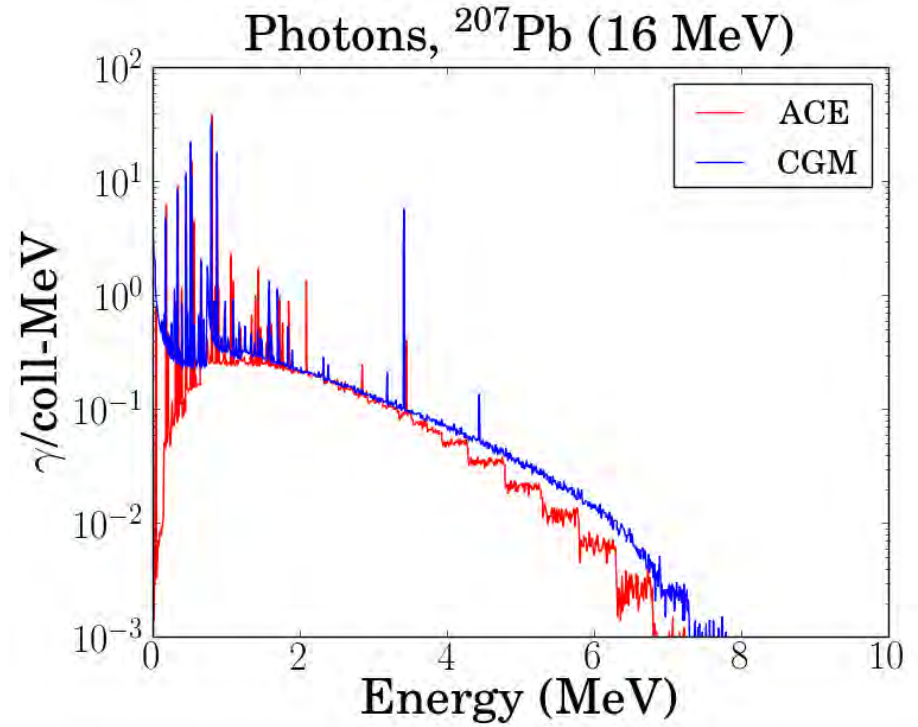
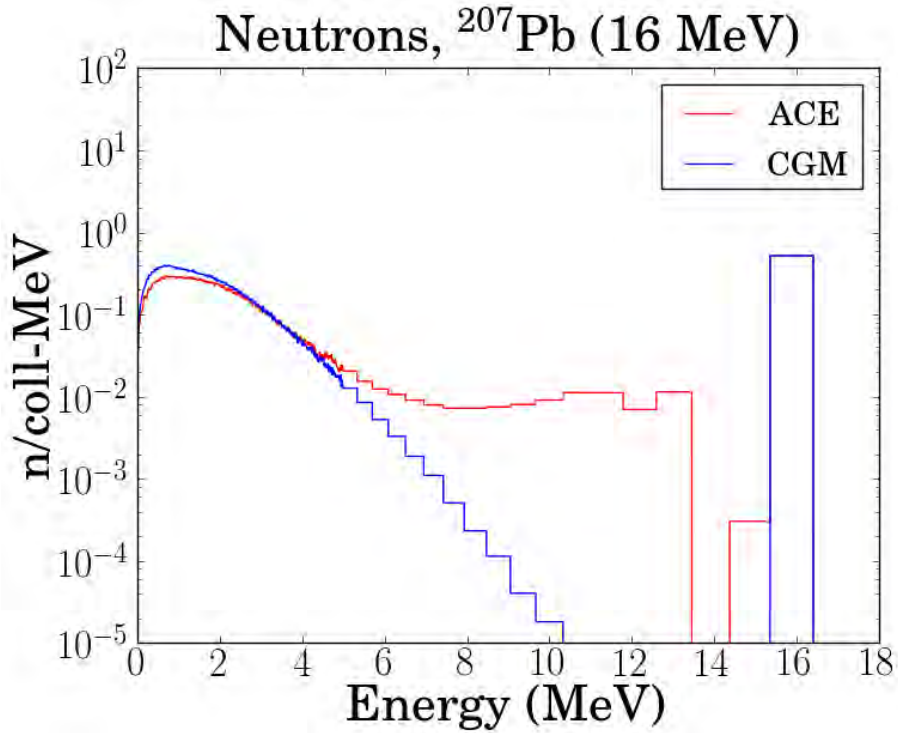
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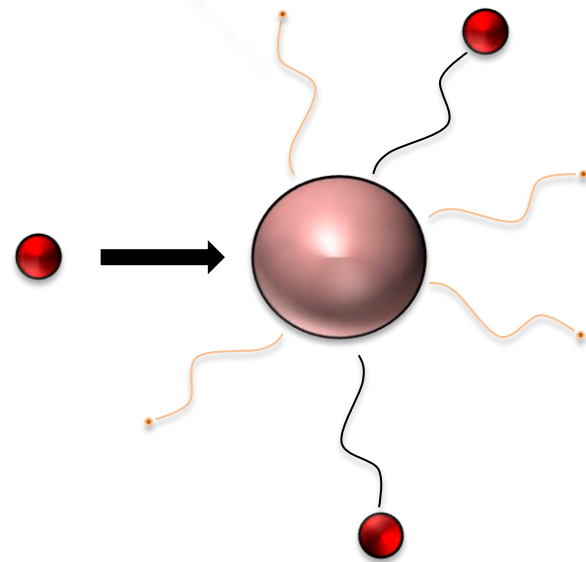
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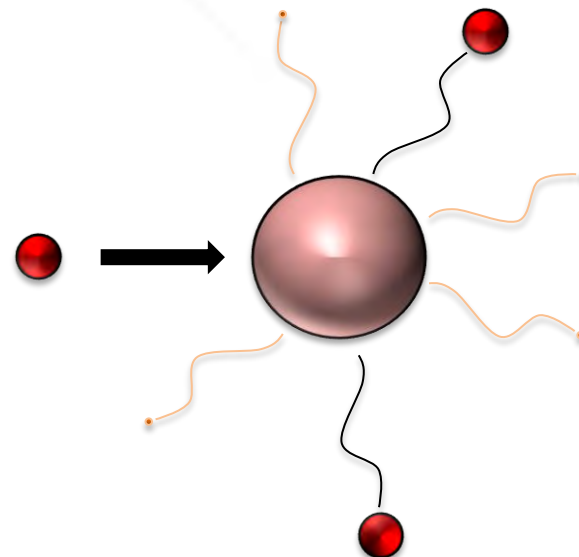
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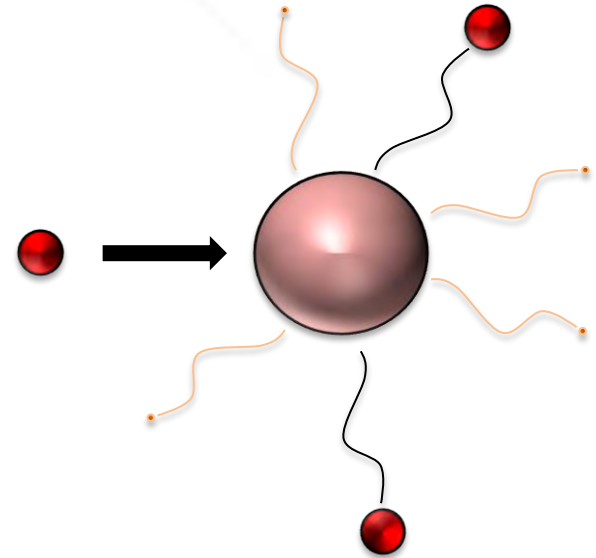
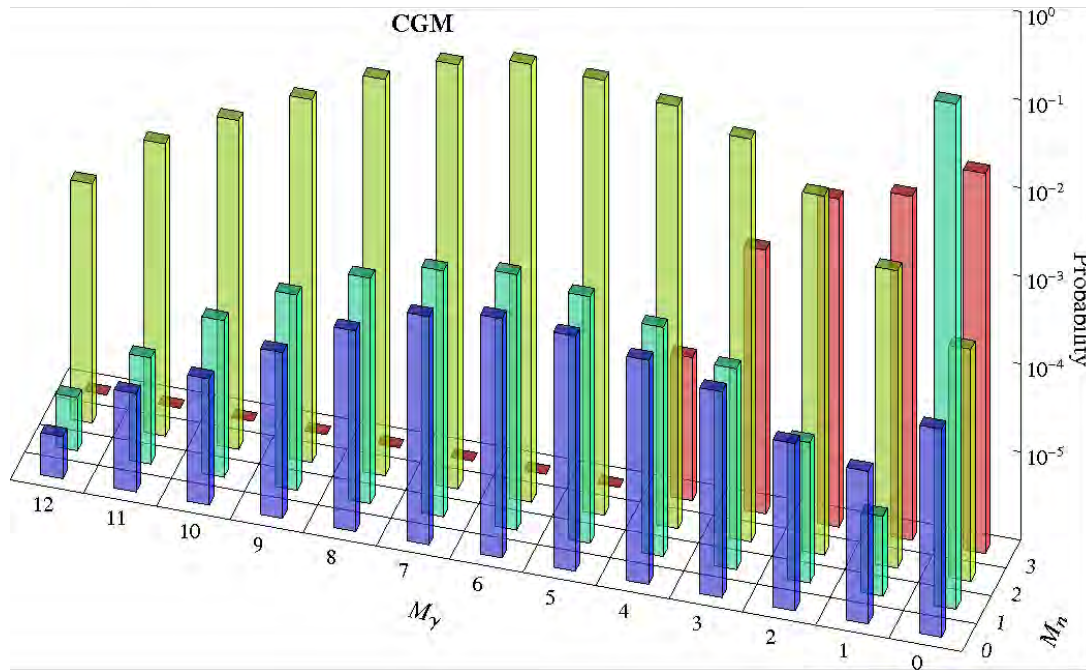
$$(n, M_n n' M_\gamma \gamma)$$



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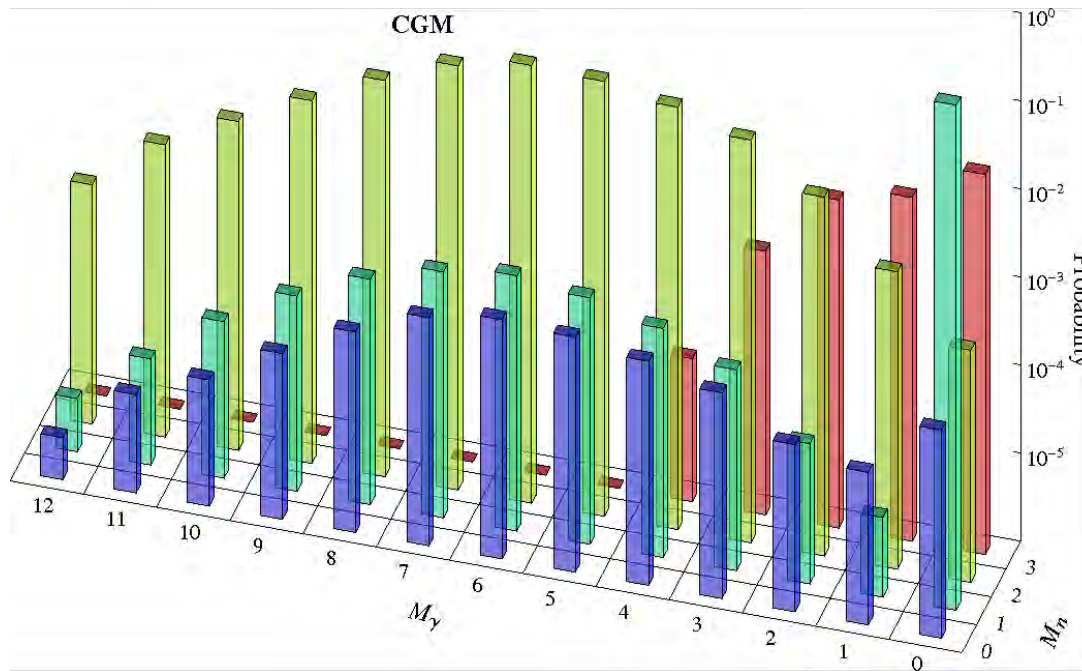
$$(n, M_n n' M_\gamma \gamma)$$



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$$(n, M_n n' M_\gamma \gamma)$$

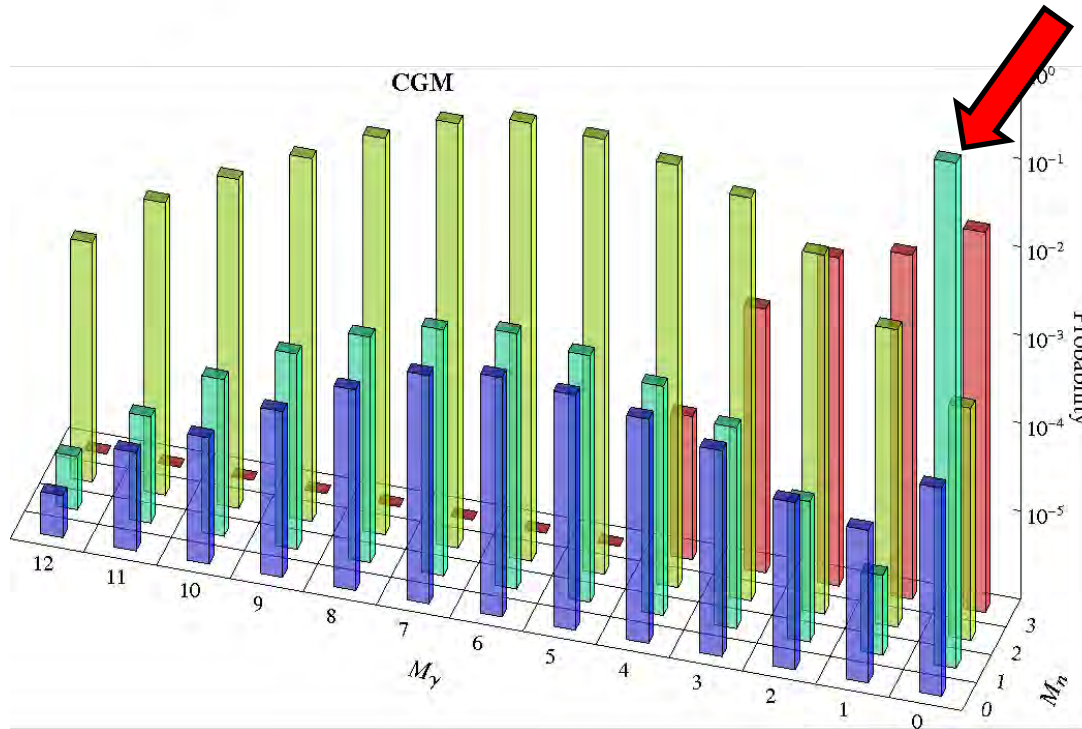


M_n	M_γ	Reaction
0	≥ 0	Capture ($n, 0n'$)
1	0	Elastic
1	> 0	Inelastic, Threshold (n, n')
> 1	≥ 0	Threshold ($n, M_n n'$)

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$$(n, M_n n' M_\gamma \gamma)$$



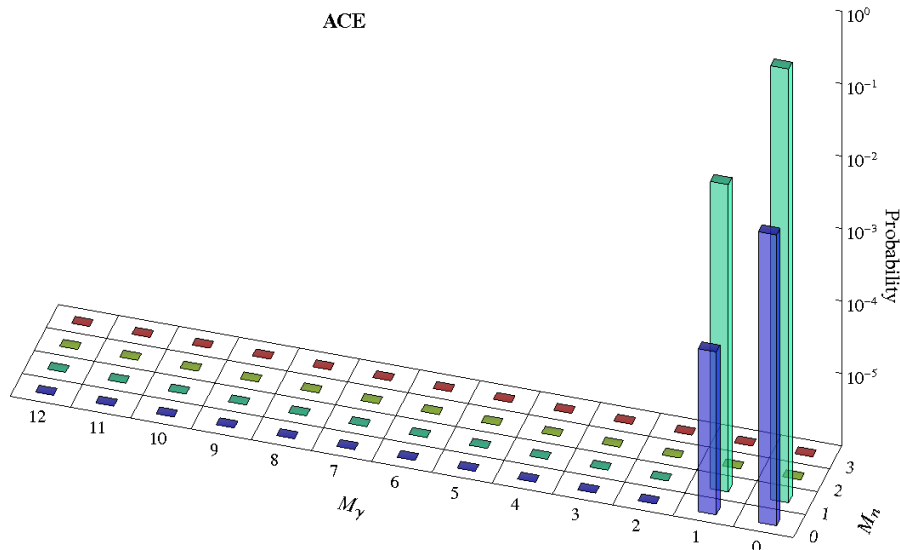
M_n	M_γ	Reaction
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> 1	≥ 0	Threshold ($n, M_n n'$)

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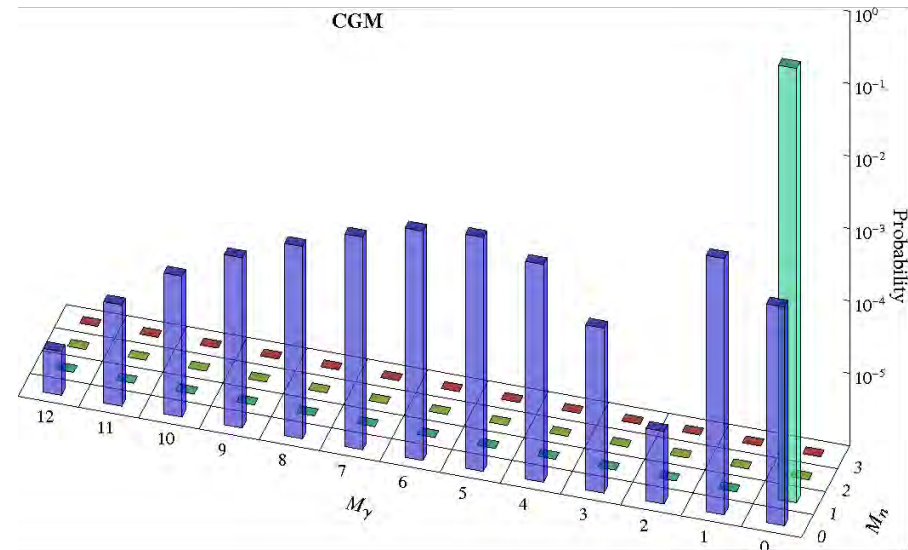
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Mat.	Energy (MeV)	<i>rxn</i>	\overline{M}_n	\overline{M}_γ
^{207}Pb	1e-6	ACE	0.99	0.02
		CGM	0.99	0.04

ACE



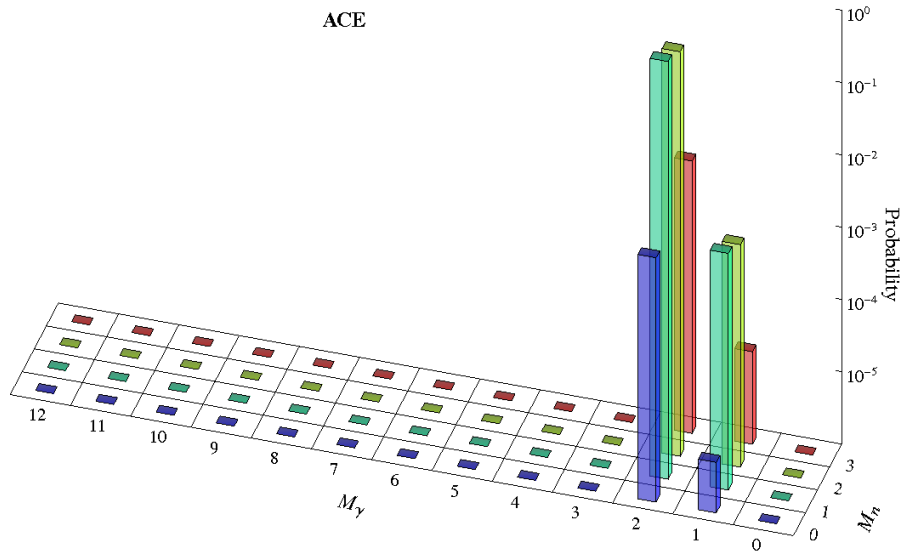
CGM



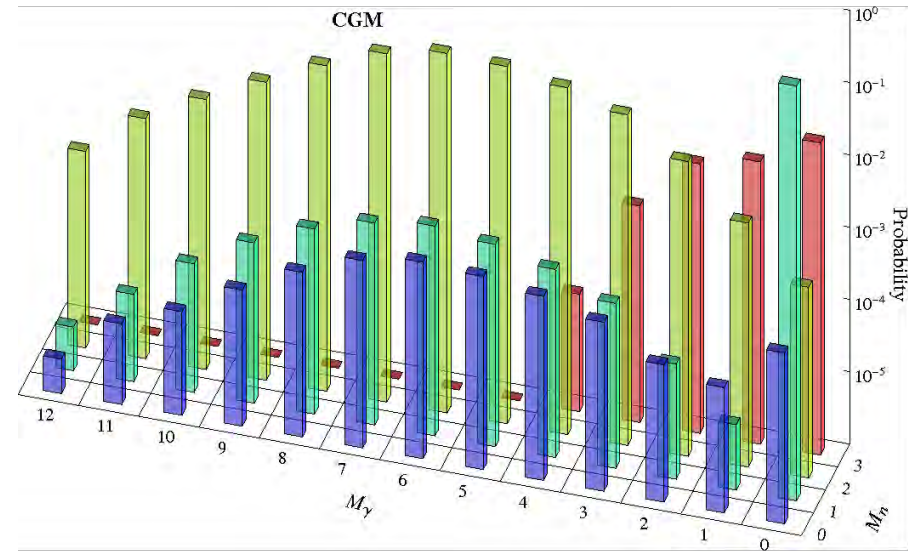
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Mat.	Energy (MeV)	<i>rxn</i>	\overline{M}_n	\overline{M}_γ
^{207}Pb	16	ACE	1.40	2.00
		CGM	1.48	2.35

ACE



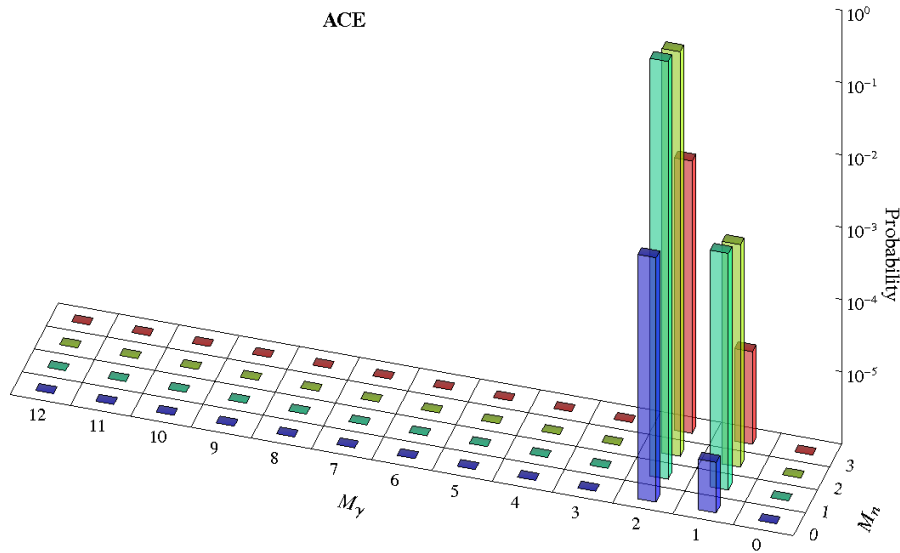
CGM



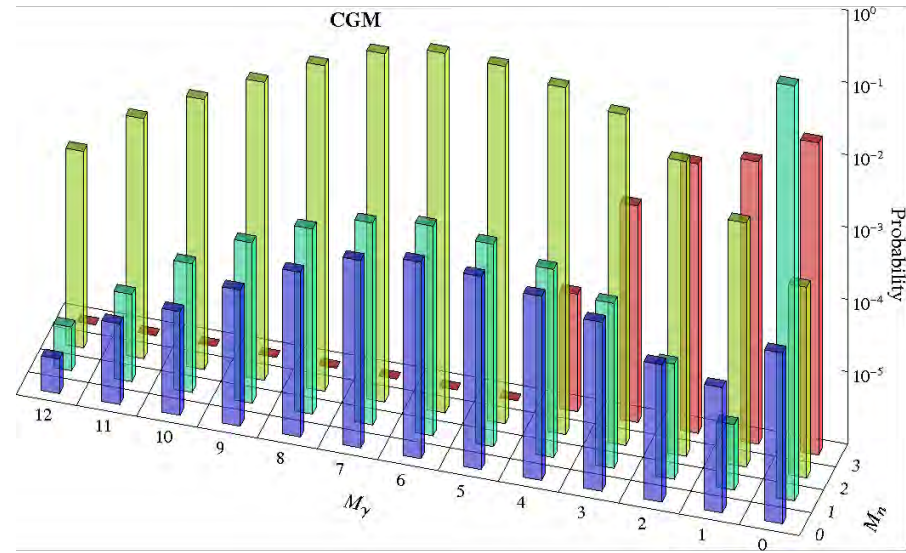
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		M_n			
		0	1	2	3
\overline{M}_γ	ACE	2.00	2.00	2.00	2.00
	CGM	5.49	0.04	5.56	0.62

ACE



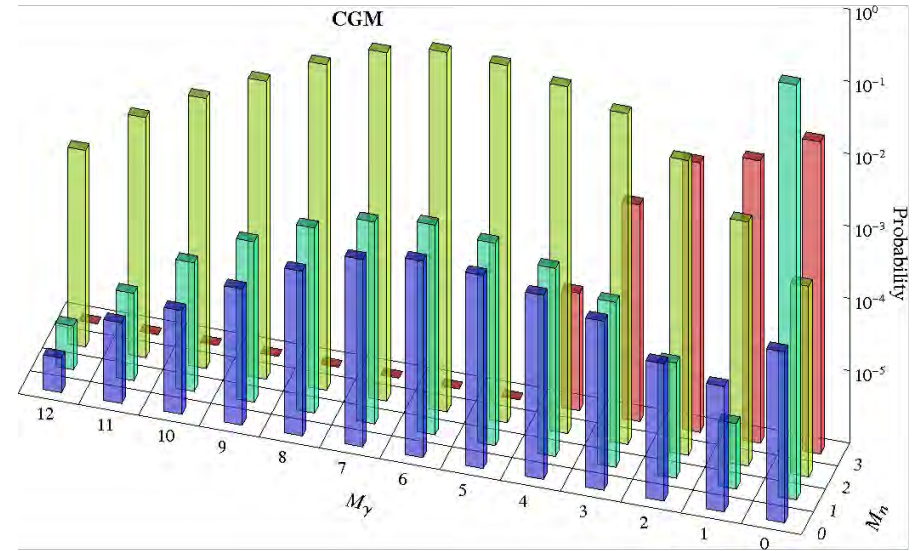
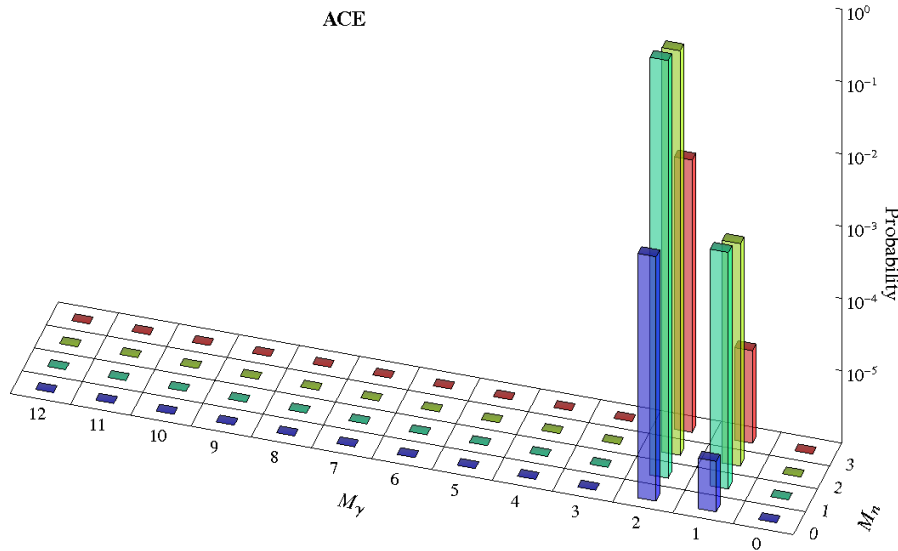
CGM



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ACE samples the same M_Y distribution or all M_n !

		M_n			
		0	1	2	3
\overline{M}_Y	ACE	2.00	2.00	2.00	2.00
	CGM	5.49	0.04	5.56	0.62



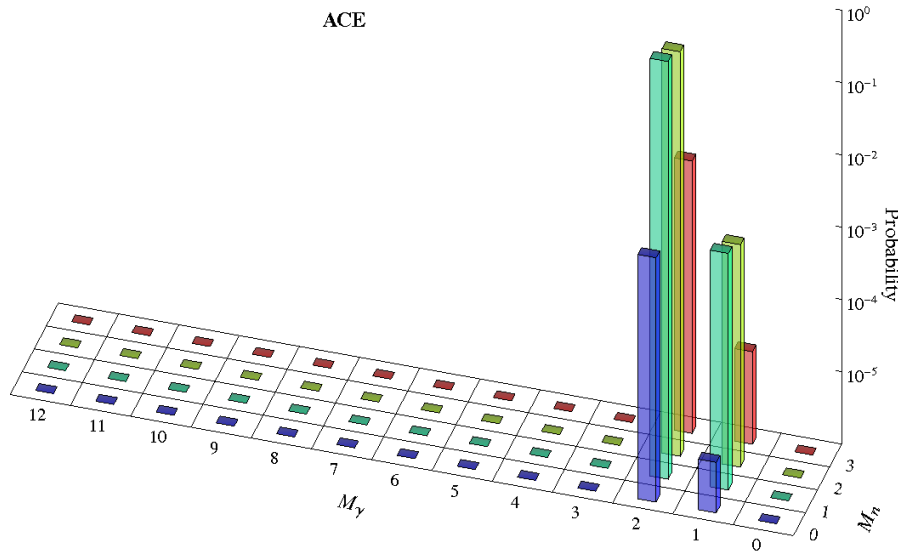
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ACE samples the same M_Y distribution or all M_n !

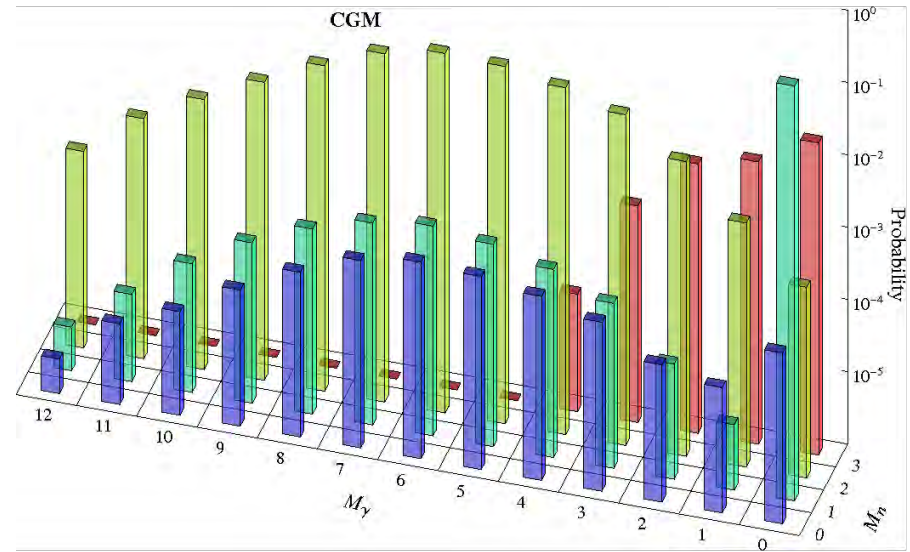
CGM has unique M_Y for each neutron multiplicity M_n !

		M_n			
		0	1	2	3
\overline{M}_Y	ACE	2.00	2.00	2.00	2.00
	CGM	5.49	0.04	5.56	0.62

ACE



CGM



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Conclusions

- CGM modified to return neutrons

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- Correlation of secondary neutron and gammas based on HF model
- Slight differences in average multiplicities between CGM/ACE

<i>rxn</i>	\overline{M}_n	\overline{M}_γ
ACE	0.75	0.88
CGM	0.75	1.03
ACE	1.22	1.18
CGM	1.47	1.33
ACE	0.99	0.02
CGM	0.99	0.04
ACE	1.40	2.00
CGM	1.48	2.35

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Conclusions

- CGM modified to return neutrons
- Provides a distribution of secondary gammas instead of binary ACE
- Correlation of secondary neutron and gammas based on HF model
- Slight differences in average multiplicities between CGM/ACE
 - Future work to resolve

<i>rxn</i>	\overline{M}_n	\overline{M}_γ
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Conclusions

- CGM modified to return neutrons
- Provides a distribution of secondary gammas instead of binary ACE
- Correlation of secondary neutron and gammas based on HF model
- Slight differences in average multiplicities between CGM/ACE
 - Future work to resolve
 - CGM suggested as theoretical model; continue utilizing ACE when average multiplicities are needed

<i>rxn</i>	\overline{M}_n	\overline{M}_γ
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References

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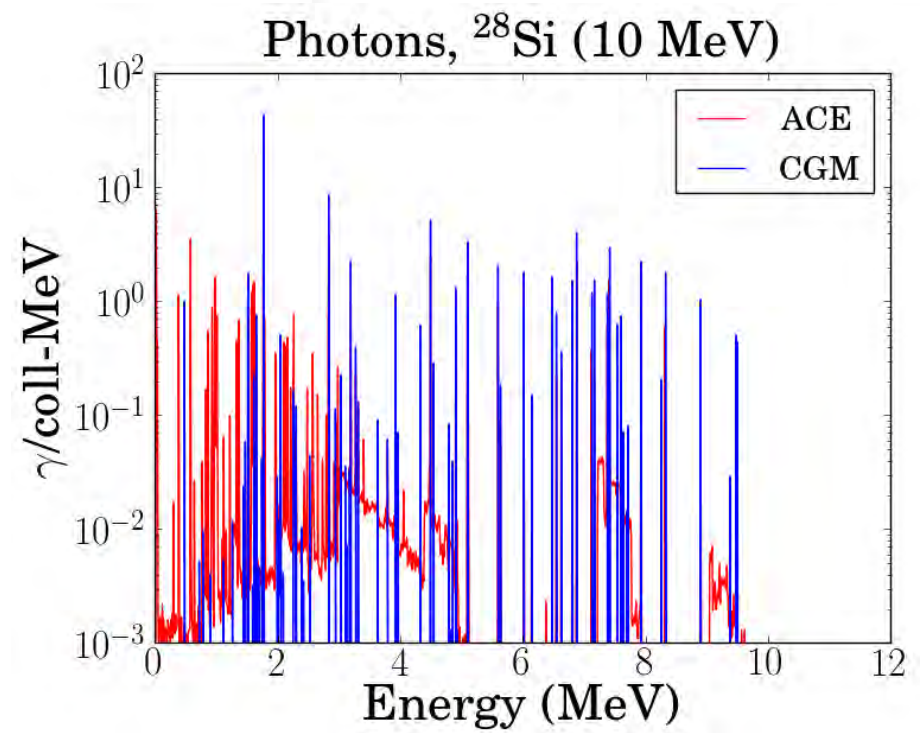
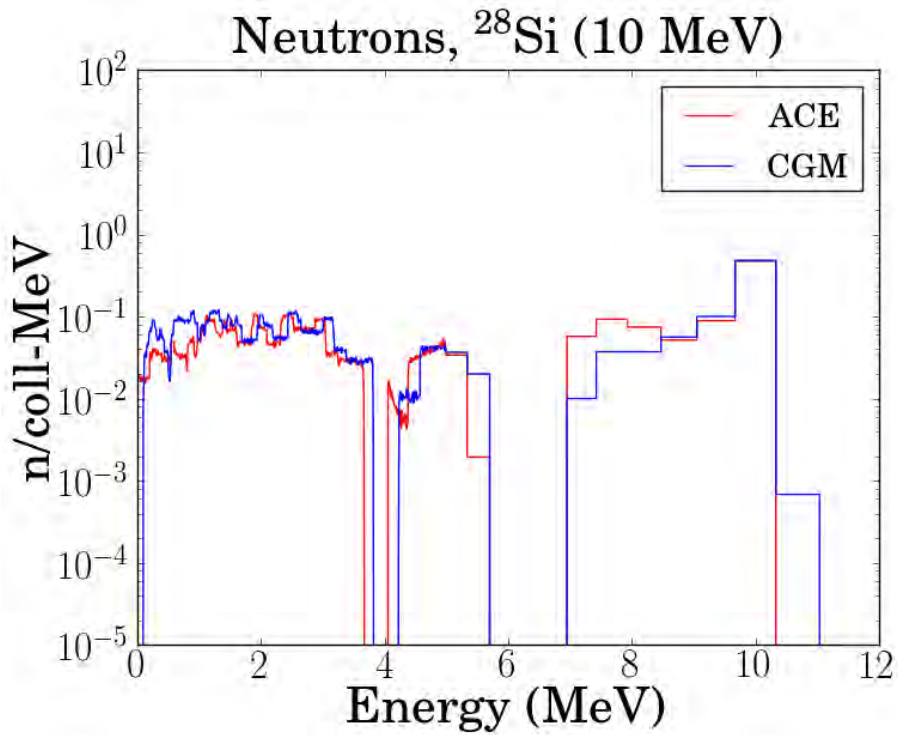
The End

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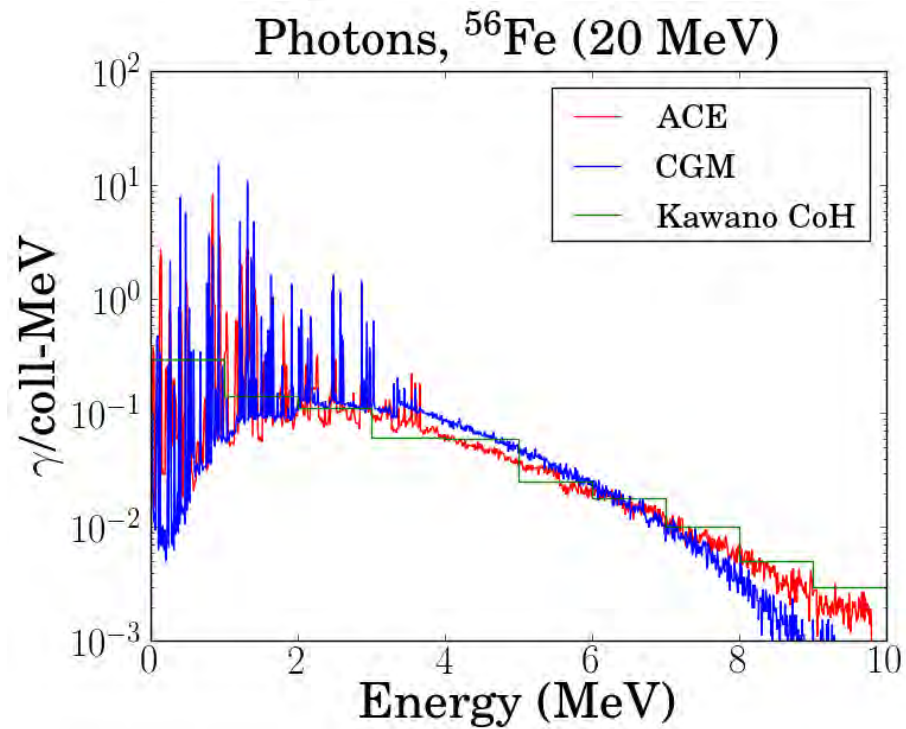
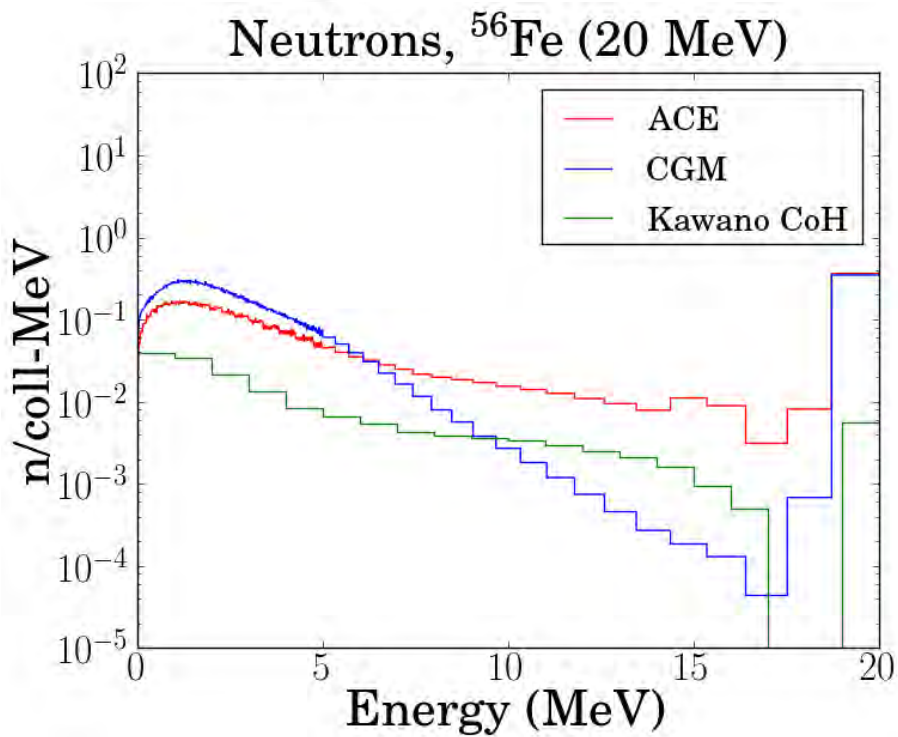
Extra Slides

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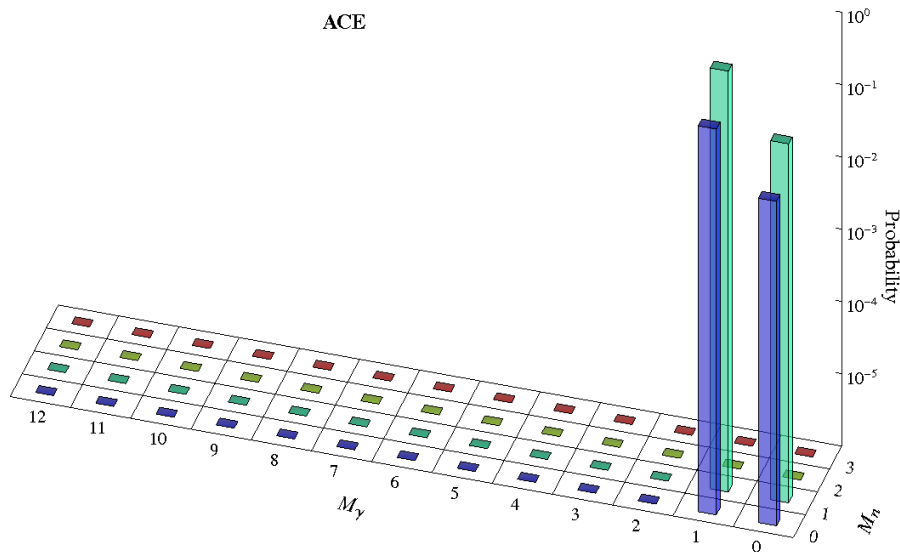


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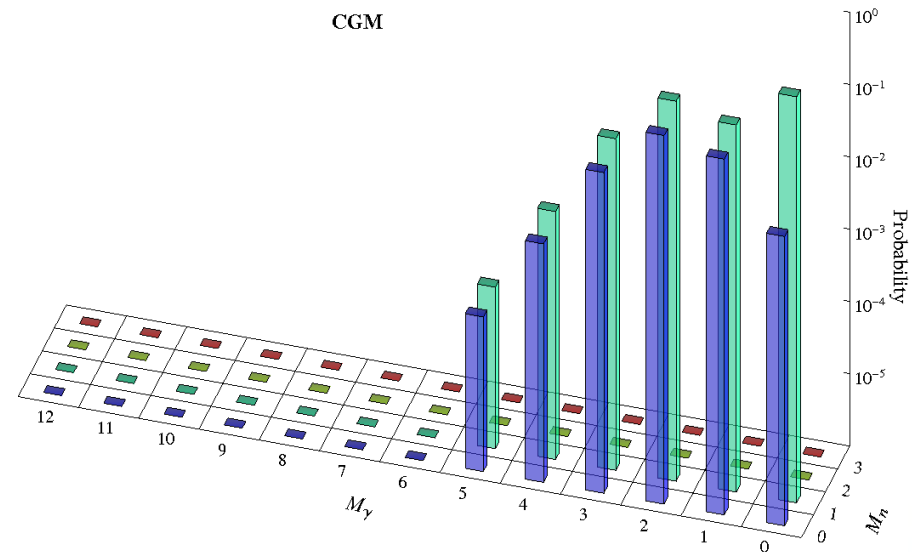
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Mat.	Energy (MeV)	<i>rxn</i>	\overline{M}_n	\overline{M}_γ
^{28}Si	10	ACE	0.75	0.88
		CGM	0.75	1.03

ACE



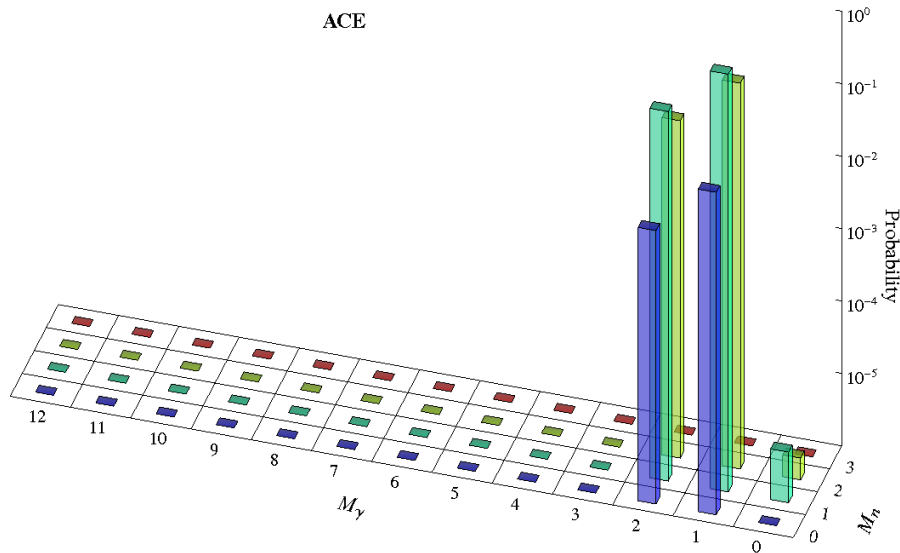
CGM



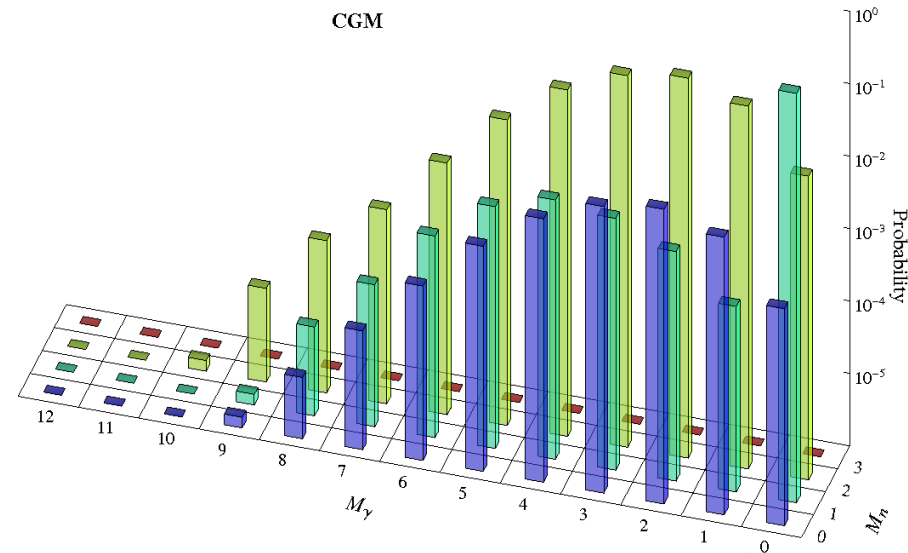
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Mat.	Energy (MeV)	<i>rxn</i>	\overline{M}_n	\overline{M}_γ
^{56}Fe	20	ACE	1.22	1.18
		CGM	1.47	1.33

ACE



CGM



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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- Secondary neutrons can additionally be sampled from a different interaction

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- Secondary neutrons can additionally be sampled from a different interaction
 - I.e., inelastic neutron with an $(n,2n')$

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- Secondary neutrons can additionally be sampled from a different interaction
 - I.e., inelastic neutron with an $(n,2n')$
- MC sampling of secondary particles can be independent of incident reaction

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Limitations to ACE Sampling

- No multiplicity distribution data for secondary particles
 - Binary sample around average
- Secondary neutrons can additionally be sampled from a different interaction
 - I.e., inelastic neutron with an $(n,2n')$
- MC sampling of secondary particles can be independent of incident reaction
 - Limits correlation of secondary particles

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